# PHYSOR–2006

**American Nuclear Society's Topical Meeting on Reactor Physics**
**Organized and hosted by the Canadian Nuclear Society**

## Advances in Nuclear Analysis and Simulation

***Sunday 2006 September 10***

Workshop Sessions:
- TRITON (ORNL)
- PARCS (Purdue University)
- DRAGON (École Polytechnique de Montréal)

***Welcome Reception***

***Monday 2006 September 11***

***Welcome***

A01 – Plenary I: Advances in Generation III+ Reactors

A02 – Plenary II: Advances in Analysis Methods

A03 – Transport Methods I

A04 – Fuel / Core Design & Analysis I

A05 – Multiprocessing Methods & Algorithms for Nuclear Applications I

A06 – Advances in Reactor Assembly & Core Analysis Methods I

***Conference Lunch***
***Guest Speaker: Dwight A. Willet, Executive Vice-President, Corporate Services, Bruce Power***

A07 – Criticality Benchmarks & Experiments I

A08 – Multiprocessing Methods & Algorithms for Nuclear Applications II

A09 – Physics & Modeling of Research Reactors I

A10 – Neutron Physics

A11 – Fuel/CoreDesign & Analysis II

A12 – Advances in Reactor Assembly & Core Analysis Methods I

A13 – Criticality Benchmarks & Experi ments II

A14 – Physics & Modeling of Research Reactors II

A15 – Transport Methods II

***Tuesday 2006 September 12***

B01 – 3-D Neutron Transport Methods I

B02 – Physics & Modeling of Research Reactors III

B03 – Uncertainty Analysis Methodologies & Applications in Reactor Safety

B04 – Nuclear Data I

B05 – Nuclear Methods for Nonproliferation & Homeland Security I

B06 – Lead-Fast-Reactor Physics

B07 – Nuclear Data II

B08 – HTR Numerical Benchmarks & Studies I

B09 – Monte-Carlo Methods & Developments I

B10 – Accelerators, Transmutation & Spallation I

B11 – Plenary III: Advances in Nuclear Data Libraries

B12 – Advanced Reactor Designs I

B13 – Reactor Physics Experiments & Analysis I

B14 – Advances in Reactor Assembly & Core Analysis Methods III

B15 – Monte-Carlo Methods & Developments II

B16 – Fuel/Core Design & Analysis III

***Dinner and Fun – Evening at Vancouver Aquarium***

***Wednesday 2006 September 13***

C01 – Covariance Data Generation for Nuclear Applications I

C02 – HTR Numerical Benchmarks & Studies II

C03 – NuclearData III

C04 – Reactor Analysis Methods I

C05 – Multi-Physics Coupled Code Systems & Multi-Scales Computation I

C06 – Covariance Data Generation for Nuclear Applications II

C07 – Accelerators, Transmutation & Spallation II

C08 – Reactor Analysis Methods II

C09 – Advanced Fuel Cycles for Fuel Management I

C10 – Advanced Reactor Design II

C11 – Advanced Fuel Cycles for Fuel Management II

C12 – Nuclear Standards

C13 – Reactor Analysis Methods III

C14 – Regulatory Perspective on Analysis & Simulation

C15 – In-Core Fuel-Management Optimization

C16 – Detector Technology

C17 – Reactor Physics Experiments & Analysis II

C18 – Nuclear Safety

C19 – Nuclear Safety Validation & Performance of ENDF/B-VII

C20 – 3-D Neutron Transport Methods II

C21 – Poster Session

***Conference Banquet***
***Guest Speaker: Dr. David Sanborn Scott, University of Victoria & Vice-President for the Americas, International Association for Hydrogen (IAHE),*** *"Can Anything Better Come Along? Reflections on the Deep Future of Hydrogen-Electricity Systems"*

***Thursday 2006 September 14***

D01 – Reactor Analysis Methods IV

D02 – Multi-Physics Coupled Code Systems & Multi-Scales Computation II

D03 – Reactor Physics I

D04 – Very High Temperature Reactor (VHTR) Physics I

D05 – OECD PBMR Benchmark

D06 – Advances in Reactor Assembly & Core Analysis Methods IV

D07 – Very High Temperature Reactor (VHTR) Physics II

D08 – Nuclear Criticality Safety

D09 – Nuclear Engineering Distance Education

D10 – Fusion Blanket Physics

D11 – International Reactor Physics Experiment Evaluation Project

D12 – Reactor Physics II

D13 – Research Reactors

D14 – International Collaboration in Reactor Physics